

#### **CTH** parameters

5 field periods		
<b>R</b> <sub>o</sub>	= 0.75 m	
a <sub>vessel</sub>	= 0.29 m	
<b>a</b> <sub>plasma</sub>	≤ 0.2m	
B <sub>o</sub>	≤ 0.7 T	
P <sub>input</sub>	≤ 30 kW ECRH	~ 200kW
Vacuum transform 0.02 – 0.35		

discharge duration ~0.1s  $n_e \le 5 \times 10^{19} \text{ m}^{-3}$ \_ ≤ 200 eV

 $OkWOH I_{D} \leq 80 kA$ <β> ≤ 0.5%

## Motivation

- Divertors isolate the confinement core from regions where the plasma and structural surfaces interact.
- Divertors in stellarators can make use of magnetic island structures at the edge of the confinement region; these structures are device dependent
- In long pulse length stellarator experiments, edge island divertors can be used as a method of plasma particle and heat exhaust.
- Knowledge of the detailed power loading on these structures and its relationship to the long connection length scrape off layer physics is a new Compact Toroidal Hybrid (CTH) research thrust.

# Overview

- Modeling results are shown for connection lengths determined by following vacuum field lines from the stagnation plane to the divertor and from those starting and terminating on the plate itself.
- For future experimental work CTH will be operated as a pure stellarator with no plasma current.
- The CTH vacuum rotational transform can be varied from t(a)=0.02 - 0.35 by adjusting the ratio of currents in the helical and toroidal field coils.
- The shaping vertical field (SVF), poloidal coil set is used to adjust the shear of the rotational transform profile, and hence the size of edge islands.
- The magnitude and phase of islands can be adjusted with a set of five error correction coils (ECC) that produce an n=1 perturbation.
- For the studies conducted, magnetic configurations with large 3,1 and 4,1 magnetic islands at the edge are generated.
- Connection lengths are shown for scans of the SVF current, ECC current, and position of the divertor plate locations relative to the island structure.

### References

[1] Y. Feng, M. Kobayashi, T. Lunt, and D. Reiter, *Plasma Phys. Control. Fusion,* **53** (2011) 024009

[2] P. Stangeby, The Plasma Boundary of Magnetic Fusion Devices, IOP Publishing (2000)

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